Modeling a tokamak from A to Z

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There is a lot going on into a plasma





There is a lot going on into a plasma (including rubbish)



Courtesy of Filippo Scotti [LLNL]



Fusion plasma physics encompasses a wide range of spatial and temporal scales





In a tokamak these scales are all coupled

Confined plasma (closed magnetic field lines)

MHD equilibrium/instabilities, microturbulence, energetic particles

Scrape-Off Layer plasma (SOL)

(open field lines)

Microturbulence, ionization, recombination radiation



The plasma is surrounded by solid structures: Plasma-material interactions

External heating

Radiofrequency waves Neutral beams

Fueling

Gas injection Pellets



Particles and energy are 'confined' by magnetic fields



The plasma is surrounded by solid structures: Plasma-material interactions

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Radiofrequency waves Neutral beams

Fueling

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and they can also 'flow' along open magnetic field lines



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Plasmas in a tokamak are in contact with the 'wall'

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Microturbulence, ionization, recombination radiation

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Plasmas in a tokamak need to be 'heated'

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Plasmas need frequent pit stops for 're-fueling'

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Modeling a tokamak is like playing with LEGO[®] ... all you need is a lot of bricks, the good ones





A tokamak simulator needs to connect fast (transport) and slow (current diffusion) time scales



A tokamak is a transformer, but the secondary circuit is a conducting fluid ... sooo complicated





A plasma in equilibrium can be described by ideal MHD

Equilibrium condition: $\mathbf{j} \times \mathbf{B} = \nabla p$ $\mathbf{B} \cdot \nabla p = 0$ $\mathbf{j} \cdot \nabla p = 0$

- ⇒ **j**, **B** lie on nested surfaces
- \Rightarrow **j**, **B**, p are described by a flux function ψ
- \Rightarrow equilibrium entirely defined by:

$$R\frac{\partial}{\partial R}\left(\frac{1}{R}\frac{\partial\psi}{\partial R}\right) + \frac{\partial^2\psi}{\partial z^2} = -\mu_0 R J_\phi$$



[Credit, DIFFER website, The Netherlands]



The first step is to get all coil currents and plasma shape right

$$R\frac{\partial}{\partial R}\left(\frac{1}{R}\frac{\partial\psi}{\partial R}\right) + \frac{\partial^{2}\psi}{\partial z^{2}} = -\mu_{0}RJ_{\phi}$$
$$= -\mu_{0}\sum_{i=1}^{N}R_{i}I_{i}\delta(R-R_{i})\delta(z-z_{i})$$

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The second step is a good model for core transport



Separation of scales enables representing (almost) any transport problem as a diffusion/convection-like problem

The goal is to obtain a set of diffusion-like equations in the form:

$$\frac{\partial Q}{\partial t} + \nabla \cdot \Gamma = S(Q, \mathbf{r}, t)$$

⇒ for a physical variable **Q** ⇒ identify the flux Γ ⇒ and the source and sink terms contained in **S**

Understanding and modeling tokamak turbulent transport requires theory-based prediction of flux-gradient relationships



6D Vlasov equations => 5D nonlinear "gyrokinetic"

State-of-the-art multi-scale ($\rho_i \rightarrow \rho_e$) ~50M CPU-hrs for 3-point scan [*N. Howard, Nucl. Fusion (2016)*]



[Citrin NF 55 (2015), Meneghini, NF 57 (2017)]



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The third step is realistic models for heating, current and momentum



External sources do more than providing heating and current: they can provide momentum and control of MHD instabilities





Perturbation of equilibrium enables description of waves propagation by representing the plasma with a dielectric tensor



[courtesy of S. Shiraiwa (PSFC), VI2.00003]



Ray-tracing equations are accurate (and fast) approximations of high frequency wave propagation





Let's put everything together ...



TRANSP is a 1.5D equilibrium and transport solver for tokamak plasma simulations developed at PPPL

TRANSP

Particle:
$$\frac{\partial}{\partial t}(nV') - \frac{\partial}{\partial \rho} \left(\left| \nabla \rho \right|^2 V' D \frac{\partial n}{\partial \rho} \right) = V' \left(S_{gas} + S_{beam} - L_{c-x, recomb, etc} \right)$$

Energy (e):
$$\frac{\partial}{\partial t} \left[\frac{3}{2} V' n_e k T_e \right] + \frac{\partial}{\partial \rho} \left[V' \left\langle \left| \nabla \rho \right|^2 \right\rangle n_e k \left(T_e v_e - \chi_e \nabla T_e \right) \right] = V' \left(P_{OH} + P_{beam} - P_{ie} - P_{rad} \right)$$

Momentum:
$$\frac{\partial}{\partial t} \left(n_i m_i V' \left\langle R^2 \right\rangle \omega \right) + \frac{\partial}{\partial \rho} \left[V' \Gamma_{\Omega} \right] = V' \left(\sum T_{iapus} - \nabla \cdot \Pi_{\phi} - mnR(\omega - \omega^*) \tau_{damp}^{-1} \right)$$

Interpretive: INPUT: $\mathbf{T}_{e}, \mathbf{T}_{i}, \mathbf{n}_{e}, \mathbf{v}_{\phi}, \dots$ OUTPUT: $\mathbf{D}_{e,i}, \chi_{e,i,\phi}, \dots$ Predictive: INPUT (model): $\chi_{e,i}, \mathbf{D}_{e,i}, \dots$ OUTPUT: $\mathbf{T}_{e,i}, \mathbf{n}_{e}, \dots$



We have learnt from modeling of ITER that the ramp-up phase is critical for sustainment of q_{min} as much as a correct choice of H&CD mix

33MW NB + 20MW EC + 20MW LH



• slower current ramp-up rate

- Early RF core heating
- => Delay current penetration and q relaxation
 - Core electron heating and current early
 - Off-axis current after H-mode formation
 - \Rightarrow Sustains bootstrap current at mid-radius
 - \Rightarrow Prevents q relaxation





Time-dependent modeling of current ramp-up has highlighted the role of RF to heat the plasma k=8->13 m k=3 -> 8 m (a) (a) HHFW P_{RF}(MW) Minimize power needs to obtain to fast ions 2 ECH the same current eleč ions 0 6 (b) (b) total

Everything works on paper ...

The challenge now is to demonstrate it in experiments.





Time-dependent application: assessment of O-X-B startup in NSTX-U

Slowdens Slowdens 1.5 1.5 0 1 EBW EBW 1 0.5 0.5 Z [m] ۲ [m] 0 -0.5 -0.5 -1 -1 -1.5 -1.51 0 2 -1 0 2 (a) (b) R [m] X [m]

Used TRANSP to assess the propagation and conversion of EC waves in NSTX-U

GOAL: find optimal launching geometry that maximizes both heating and current that does not make the plasma unstable

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[N. Lopez (Princeton University) PPCF 2016]





Used integrated modeling to design experiments BEFORE going to the control room

- Feed-forward NBI and EC predicted first with TRANSP (no feedback)
- Improved access to high beta with little MHD



What is missing?



What happens close to the plasma wall is very important



Edge transport and fuelling are critical ingredients to model the plasma evolution in burning plasma conditions



Heat conduction zone

Impurity radiation zone

H⁰/D⁰/T⁰ ionization zone (Te>5eV)

Neutral friction zone

Recombination zone (Te<1eV)



Courtesy of R. Pitts (ITER Organization)



Hybrid approach to modeling of RF wave propagation is a promising avenue towards implementation in tokamak simulator



Core: Axisymmetric flux surface grid Hot plasma conductivity Dense Matrix Solver Edge: Unstructured mesh with complicated geometry (either 2D or 3D) Cold plasma with collision.

Boundary: matching technique to build integrated solution



Courtesy of S. Shiraiwa (PSFC)

Would benefit from realistic model of SOL



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Integrated Tokamak Modeling offers plenty of opportunities to understand and model experiments (we have collaborations all over the world)

... and it is a lot of FUN !

JOIN OUR TEAM fpoli@pppl.gov

